

Analysis of Materials for MEBT Absorber

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Abstract: Neutron yields from copper and molybdenum MEBT absorber are compared.

Input data

The present favorite material is for the MEBT absorber is TZM alloy:

Ti 0.55% Max
Zr 0.12% Max
Mo Balance

The beam parameters: 2.1 MeV H-, 10 mA CW.

Threshold for the possible reactions [1]

Data for **Mo**:

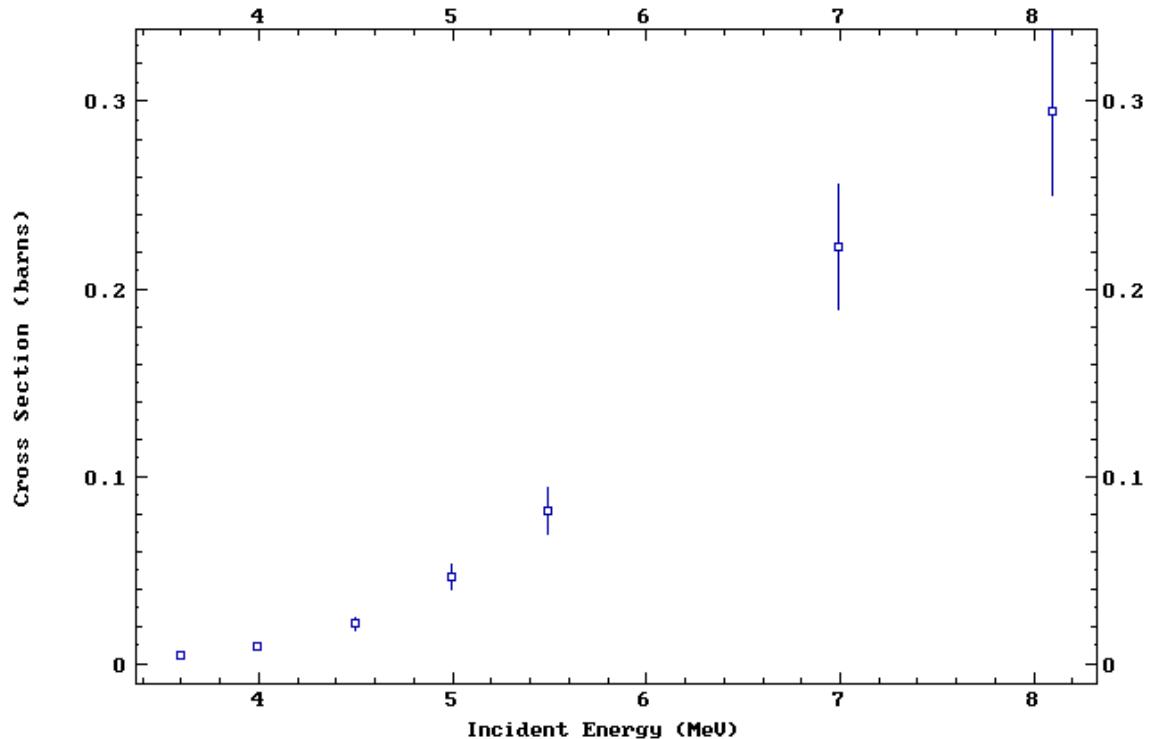
Isotope	Abundance, %	Reaction	Threshold, MeV
⁹⁸ Mo	24.13	⁹⁸ Mo (pn) ⁹⁸ Tc	2.492
⁹⁶ Mo	16.68	⁹⁶ Mo (pn) ⁹⁶ Tc	3.795
⁹⁵ Mo	15.92	⁹⁵ Mo (pn) ⁹⁵ Tc	2.499
⁹² Mo	14.84	⁹² Mo (pn) ⁹² Tc	8.748
¹⁰⁰ Mo	9.63	¹⁰⁰ Mo (pn) ¹⁰⁰ Tc	0.960
		¹⁰⁰ Tc (pn) ¹⁰⁰ Ru	0.0
		¹⁰⁰ Ru (pn) ¹⁰⁰ Rh	4.462
⁹⁷ Mo	9.55	⁹⁷ Mo (pn) ⁹⁷ Tc	1.114
		⁹⁷ Tc (pn) ⁹⁷ Ru	1.910
		⁹⁷ Ru (pn) ⁹⁷ Rh	4.350
⁹⁴ Mo	9.25	⁹⁴ Mo (pn) ⁹⁴ Tc	5.092

So, isotopic composition of residual:

Isotope	Abundance, %	Final Isotope
⁹⁸ Mo	24.13	⁹⁸ Mo
⁹⁶ Mo	16.68	⁹⁶ Mo
⁹⁵ Mo	15.92	⁹⁵ Mo
⁹² Mo	14.84	⁹² Mo
¹⁰⁰ Mo	9.63	\rightarrow ¹⁰⁰ Tc \rightarrow ¹⁰⁰ Ru
⁹⁷ Mo	9.55	\rightarrow ⁹⁷ Tc \rightarrow ⁹⁷ Ru
⁹⁴ Mo	9.25	⁹⁴ Mo

Cross sections [2]

42-MO-0(P,N)
EXFOR Request: 8404/1, 2012-Jan-10 21:34:07



```
#ZView-data-copy: 10-Jan-2012 21:34:07
=====
#
#name: 42-MO-0(P,N),,SIG
#X.axis: Incident Energy
#Y.axis: Cross Section
#wdata: 3
#ldata: 7
```

```

#data...
#
#      X          Y          +-dY  # Comments...
#      MeV        barns     barns  # Year,Author(s)    ## EXFOR-ID
#
#      3.6        0.005    0.00075 # 1959,R.D.Albert ## T0130015
#      4          0.01     0.0015   # 1959,R.D.Albert ## T0130015
#      4.5        0.022    0.0033   # 1959,R.D.Albert ## T0130015
#      5          0.047    0.00705 # 1959,R.D.Albert ## T0130015
#      5.5        0.082    0.0123   # 1959,R.D.Albert ## T0130015
#      7          0.223    0.03345 # 1959,R.D.Albert ## T0130015
#      8.1        0.295    0.04425 # 1959,R.D.Albert ## T0130015
//
#-----

```

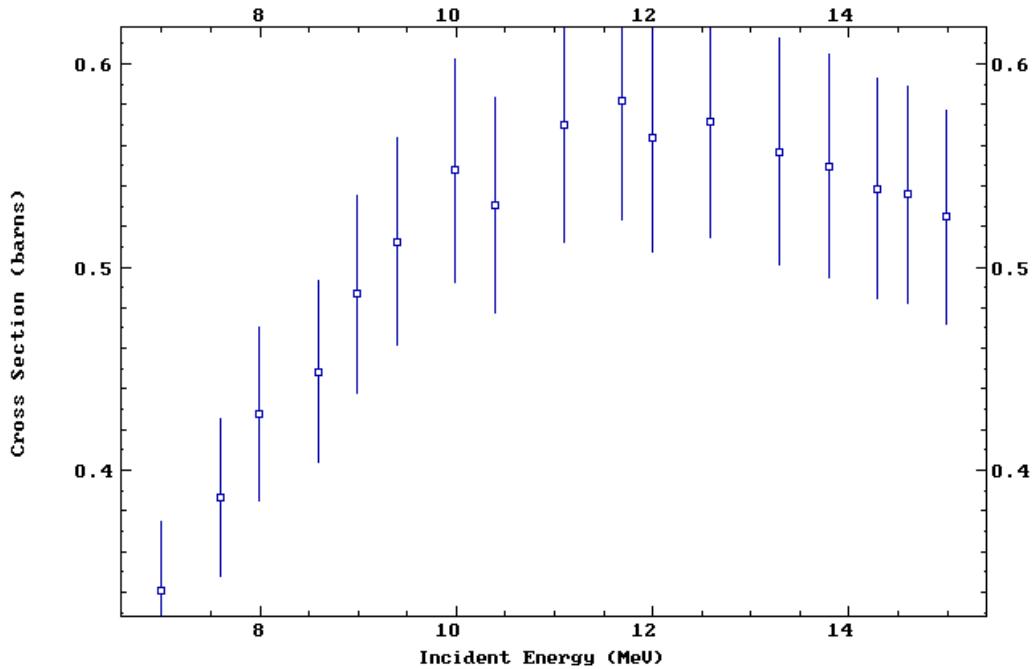
Data for **Ti**:

Isotope	Abundance,%	Reaction	Threshold, MeV
⁴⁸ Ti	73.80	⁴⁸ Ti (pn) ⁴⁸ V	4.895
⁴⁶ Ti	8.00	⁴⁶ Ti (pn) ⁴⁶ V	8.005
⁴⁷ Ti	7.30	⁴⁷ Ti (pn) ⁴⁷ V	3.792
⁴⁹ Ti	5.50	⁴⁹ Ti (pn) ⁴⁹ V ⁴⁹ V (pn) ⁴⁹ Cr	1.413 3.479
⁵⁰ Ti	5.40	⁵⁰ Ti (pn) ⁵⁰ V	3.048

So, isotopic composition of residual:

Isotope	Abundance,%	Final Isotope
⁴⁸ Ti	73.80	⁴⁸ Ti
⁴⁶ Ti	8.00	⁴⁶ Ti
⁴⁷ Ti	7.30	⁴⁷ Ti
⁴⁹ Ti	5.50	→ V
⁵⁰ Ti	5.40	⁵⁰ Ti

22-TI-0(P,N)
EXFOR Request: 8403/1, 2012-Jan-10 21:27:16



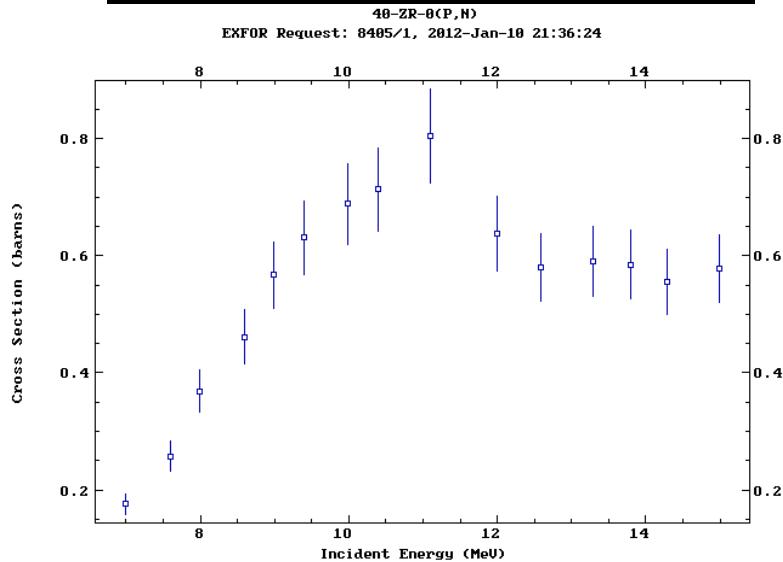
```
#ZView-data-copy: 10-Jan-2012 21:27:16
=====
#
#name: 22-TI-0(P,N),,SIG
#X.axis: Incident Energy
#Y.axis: Cross Section
#wdata: 3
#ldata: 17
#data...
#      X          Y      +-dY # Comments...
#    MeV        barns     barns # Year,Author(s)    ## EXFOR-ID
7       0.341      0.0341 # 1967,G.Chodil+ ## C0693003
7.6     0.387      0.0387 # 1967,G.Chodil+ ## C0693003
8       0.428      0.0428 # 1967,G.Chodil+ ## C0693003
8.6     0.449      0.0449 # 1967,G.Chodil+ ## C0693003
9       0.487      0.0487 # 1967,G.Chodil+ ## C0693003
9.4     0.513      0.0513 # 1967,G.Chodil+ ## C0693003
10      0.548      0.0548 # 1967,G.Chodil+ ## C0693003
10.4    0.531      0.0531 # 1967,G.Chodil+ ## C0693003
11.1    0.57       0.057   # 1967,G.Chodil+ ## C0693003
11.7    0.582      0.0582 # 1967,G.Chodil+ ## C0693003
12      0.564      0.0564 # 1967,G.Chodil+ ## C0693003
12.6    0.572      0.0572 # 1967,G.Chodil+ ## C0693003
13.3    0.557      0.0557 # 1967,G.Chodil+ ## C0693003
13.8    0.55       0.055   # 1967,G.Chodil+ ## C0693003
14.3    0.539      0.0539 # 1967,G.Chodil+ ## C0693003
14.6    0.536      0.0536 # 1967,G.Chodil+ ## C0693003
15      0.525      0.0525 # 1967,G.Chodil+ ## C0693003
//
```

Data for **Zr**:

Isotope	Abundance, %	Reaction	Threshold, MeV
⁹⁰ Zr	51.45	⁹⁰ Zr (pn) ⁹⁰ Nb	6.971
⁹⁴ Zr	17.38	⁹⁴ Zr (pn) ⁹⁴ Nb	1.703
		⁹⁴ Nb (pn) ⁹⁴ Mo	0.0
		⁹⁴ Mo (pn) ⁹⁴ Tc	5.092
⁹² Zr	17.15	⁹¹ Zr (pn) ⁹¹ Nb	2.818
⁹¹ Zr	11.22	⁹¹ Zr (pn) ⁹¹ Nb	2.063
		⁹¹ Nb (pn) ⁹¹ Mo	5.264
⁹⁶ Zr	2.80	⁹⁶ Zr (pn) ⁹⁶ Nb	0.628
		⁹⁶ Nb (pn) ⁹⁶ Mo	0.0
		⁹⁶ Mo (pn) ⁹⁶ Tc	3.795

So, isotopic composition of residual:

Isotope	Abundance, %	Final Isotope
⁹⁰ Zr	51.45	⁹⁰ Zr
⁹⁴ Zr	17.38	\rightarrow^{94} Nb \rightarrow^{94} Mo
⁹² Zr	17.15	⁹² Zr
⁹¹ Zr	11.22	\rightarrow^{91} Nb
⁹⁶ Zr	2.80	\rightarrow^{96} Nb \rightarrow^{96} Mo



```

#ZVView-data-copy: 10-Jan-2012 21:36:24
=====
#
#name: 40-ZR-0(P,N),,SIG
#X.axis: Incident Energy
#Y.axis: Cross Section
#wdata: 3
#ldata: 15
#data...
#      X          Y          +-dY # Comments...
#    MeV        barns      barns # Year,Author(s)    ## EXFOR-ID
#      7          0.177     0.0177 # 1967,G.Chodil+ ## C0693006
#     7.6         0.258     0.0258 # 1967,G.Chodil+ ## C0693006
#      8          0.37      0.037 # 1967,G.Chodil+ ## C0693006
#     8.6         0.462     0.0462 # 1967,G.Chodil+ ## C0693006
#      9          0.568     0.0568 # 1967,G.Chodil+ ## C0693006
#     9.4         0.632     0.0632 # 1967,G.Chodil+ ## C0693006
#     10          0.69      0.069 # 1967,G.Chodil+ ## C0693006
#    10.4         0.714     0.0714 # 1967,G.Chodil+ ## C0693006
#    11.1         0.805     0.0805 # 1967,G.Chodil+ ## C0693006
#     12          0.639     0.0639 # 1967,G.Chodil+ ## C0693006
#    12.6         0.581     0.0581 # 1967,G.Chodil+ ## C0693006
#    13.3         0.591     0.0591 # 1967,G.Chodil+ ## C0693006
#    13.8         0.586     0.0586 # 1967,G.Chodil+ ## C0693006
#    14.3         0.557     0.0557 # 1967,G.Chodil+ ## C0693006
#     15          0.579     0.0579 # 1967,G.Chodil+ ## C0693006
//
```

Data for **Cu** (for future reference):

Isotope	Abundance,%	Reaction	Threshold, MeV
⁶³ Cu	69.17	⁶³ Cu (pn) ⁶³ Zn	4.215
⁶⁵ Cu	30.83	⁶⁵ Cu (pn) ⁶⁵ Zn	2.167

So, isotopic composition of residual:

Isotope	Abundance,%	Final Isotope
⁶³ Cu	69.17	⁶³ Cu
⁶⁵ Cu	30.83	⁶⁵ Cu

```

#ZVView-data-copy: 18-Jan-2012 23:36:28
=====
#
#name: 29-CU-0(P,N),,SIG
#X.axis: Incident Energy
#Y.axis: Cross Section
#wdata: 3

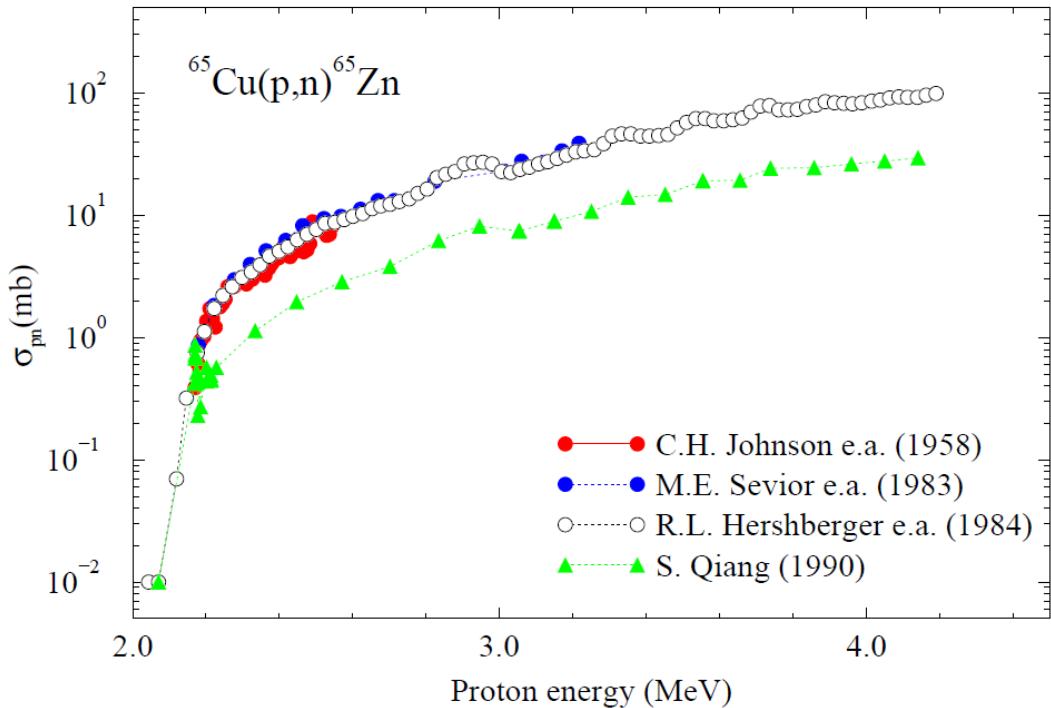
```

```

#1data: 84
#data...
#      X          Y          +-dY # Comments...
#      MeV        barns      barns # Year,Author(s)      ## EXFOR-ID
1.917    2e-5     7.4e-7 # 1990,S.Qiang      ## C0739002
1.927    0         0         # 1990,S.Qiang      ## C0739002
2.07     5e-5     9.15e-7 # 1990,S.Qiang      ## C0739002
2.07     1e-5     7.34e-7 # 1990,S.Qiang      ## C0739002
2.169    0.00042   1.6212e-5 # 1990,S.Qiang      ## C0739002
2.17     0.00068   1.6184e-5 # 1990,S.Qiang      ## C0739002
2.171    0.00073   1.4965e-5 # 1990,S.Qiang      ## C0739002
2.171    0.00087   1.479e-5 # 1990,S.Qiang      ## C0739002
2.172    0.0007    1.246e-5 # 1990,S.Qiang      ## C0739002
2.175    0.00052   8.684e-6 # 1990,S.Qiang      ## C0739002
2.178    0.00043   6.622e-6 # 1990,S.Qiang      ## C0739002
2.178    0.00023   5.221e-6 # 1990,S.Qiang      ## C0739002
2.185    0.00044   5.236e-6 # 1990,S.Qiang      ## C0739002
2.185    0.00027   4.239e-6 # 1990,S.Qiang      ## C0739002
2.202    0.00057   4.275e-6 # 1990,S.Qiang      ## C0739002
2.202    0.00044   3.784e-6 # 1990,S.Qiang      ## C0739002
2.214    0.00046   3.312e-6 # 1990,S.Qiang      ## C0739002
2.214    0.00049   3.381e-6 # 1990,S.Qiang      ## C0739002
2.214    0.00045   3.285e-6 # 1990,S.Qiang      ## C0739002
2.214    0.00046   3.266e-6 # 1990,S.Qiang      ## C0739002
2.214    0.00045   3.24e-6 # 1990,S.Qiang      ## C0739002
2.229    0.00057   3.192e-6 # 1990,S.Qiang      ## C0739002
2.334    0.00113   3.955e-6 # 1990,S.Qiang      ## C0739002
2.447    0.00196   5.292e-6 # 1990,S.Qiang      ## C0739002
2.57     0.00287   8.897e-6 # 1990,S.Qiang      ## C0739002
2.701    0.0038    1.026e-5 # 1990,S.Qiang      ## C0739002
2.834    0.00616   1.2936e-5 # 1990,S.Qiang      ## C0739002
2.946    0.00816   1.9584e-5 # 1990,S.Qiang      ## C0739002
3.053    0.0074    1.85e-5 # 1990,S.Qiang      ## C0739002
3.054    0.00742   1.855e-5 # 1990,S.Qiang      ## C0739002
3.149    0.00893   2.5004e-5 # 1990,S.Qiang      ## C0739002
3.149    0.00892   2.4976e-5 # 1990,S.Qiang      ## C0739002
3.251    0.01071   2.7846e-5 # 1990,S.Qiang      ## C0739002
3.351    0.01409   3.0998e-5 # 1990,S.Qiang      ## C0739002
3.452    0.01475   3.245e-5 # 1990,S.Qiang      ## C0739002
3.554    0.01907   3.6233e-5 # 1990,S.Qiang      ## C0739002
3.554    0.01913   3.6347e-5 # 1990,S.Qiang      ## C0739002
3.655    0.01939   5.2353e-5 # 1990,S.Qiang      ## C0739002
3.739    0.02415   5.796e-5 # 1990,S.Qiang      ## C0739002
3.858    0.02444   5.8656e-5 # 1990,S.Qiang      ## C0739002
3.959    0.02637   6.0651e-5 # 1990,S.Qiang      ## C0739002
4.05     0.02777   6.1094e-5 # 1990,S.Qiang      ## C0739002
4.141    0.02952   6.4944e-5 # 1990,S.Qiang      ## C0739002
4.231    0.03678   6.9882e-5 # 1990,S.Qiang      ## C0739002
4.323    0.04715   8.0155e-5 # 1990,S.Qiang      ## C0739002
4.323    0.04734   8.0478e-5 # 1990,S.Qiang      ## C0739002
4.323    0.04719   8.0223e-5 # 1990,S.Qiang      ## C0739002
4.412    0.05565   0.00012243 # 1990,S.Qiang      ## C0739002
4.44     0.05876   0.0001234 # 1990,S.Qiang      ## C0739002
4.44     0.05875   0.00012337 # 1990,S.Qiang      ## C0739002
4.51     0.06964   0.00013928 # 1990,S.Qiang      ## C0739002

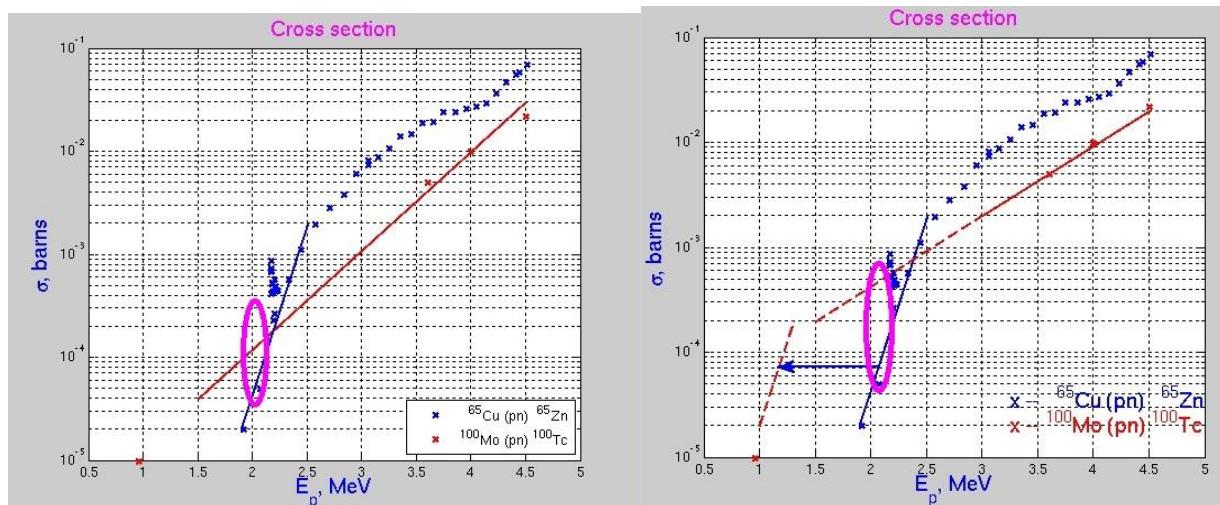
```

Experimental data for cross section [3]:



Cross Section Analysis

1. In the production of neutrons it can be neglected of the contribution for all reactions except $^{100}\text{Mo}(\text{pn})^{100}\text{Tc}$ and $^{97}\text{Mo}(\text{pn})^{97}\text{Tc}$. Overall abundance for these isotopes is $\eta \leq 20\%$.
2. For these reactions it is possible to compare their cross sections with cross section of the reaction $^{65}\text{Cu}(\text{pn})^{65}\text{Zn}$ ($\eta \leq 30\%$):

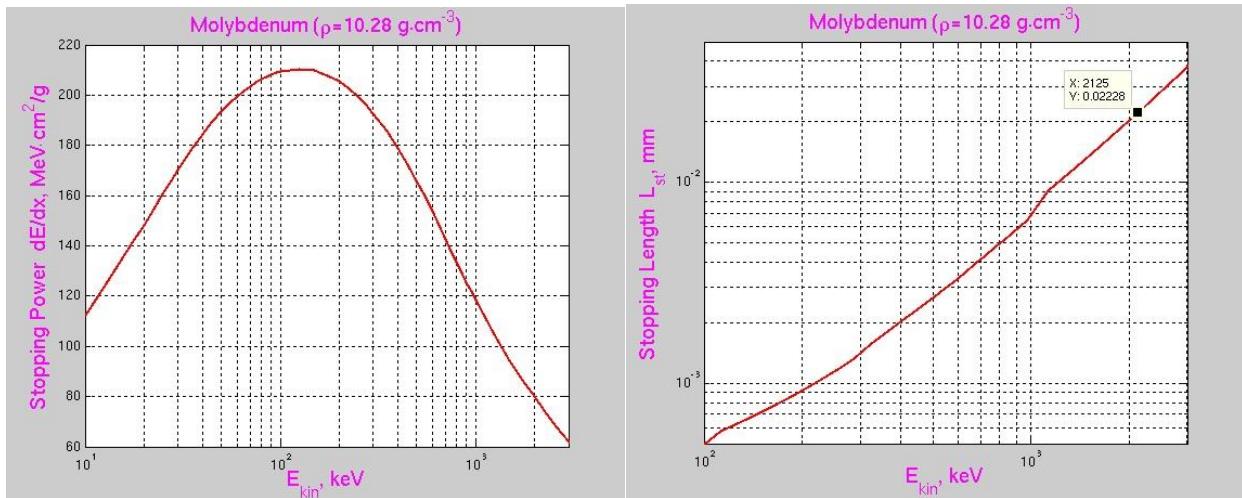


3. From these plots one can find (using different approaches) that for $E_p \approx 2.1 \text{ MeV}$ the cross sections for Mo larger than for Cu from 2 to 8 times:

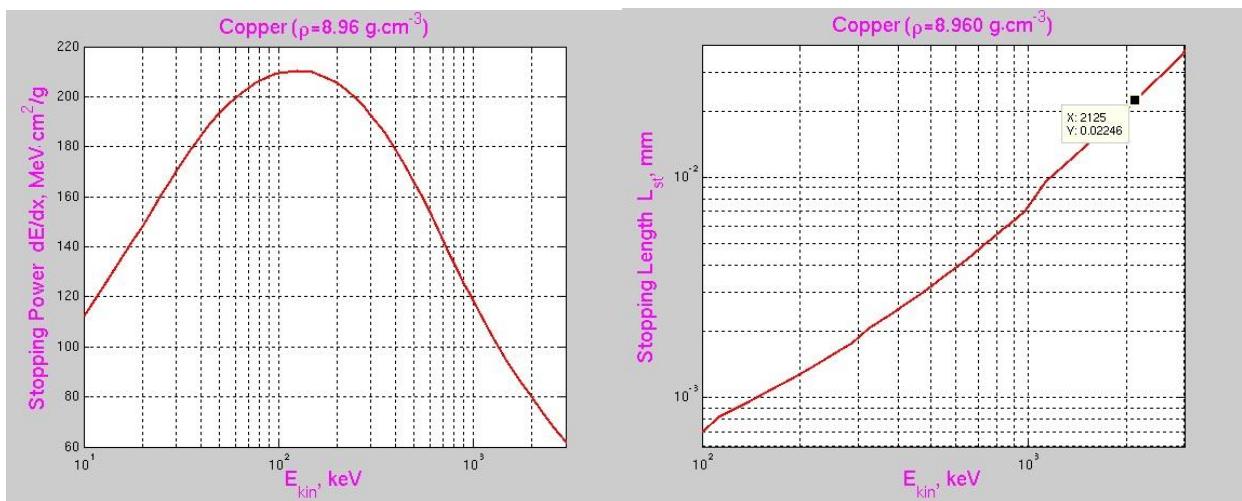
$$\sigma \approx \begin{cases} 0.12 - 0.50 \text{ mb} & \text{for Mo;} \\ 0.06 \text{ mb} & \text{for Cu.} \end{cases}$$

Stopping Length Data

Using data from [4] one can find the stopping powers for protons (with energy $E_p \approx 2.1 \text{ MeV}$) and their stopping lengths in molybdenum and copper:



Molybdenum



Copper

From these data one have the following results:

$$\lambda_{st} \approx 2.2 \cdot 10^{-3} \text{ cm for Cu and Mo.}$$

Neutron Yield

Let's calculate the neutron yield per proton:

$$N_n = 6.02 \cdot 10^{23} \frac{\rho}{A} \eta \sigma \lambda_{st},$$

For input data

	A	$\rho, \text{ g} \cdot \text{cm}^{-3}$	$\sigma, \text{ mb}$	$\eta, \%$	$\lambda_{st}, \text{ cm}$
Mo	95.94	10.28	0.12 ÷ 0.50	20	$2.2 \cdot 10^{-3}$
Cu	63.54	8.94	0.06	30	$2.2 \cdot 10^{-3}$

one has the following results:

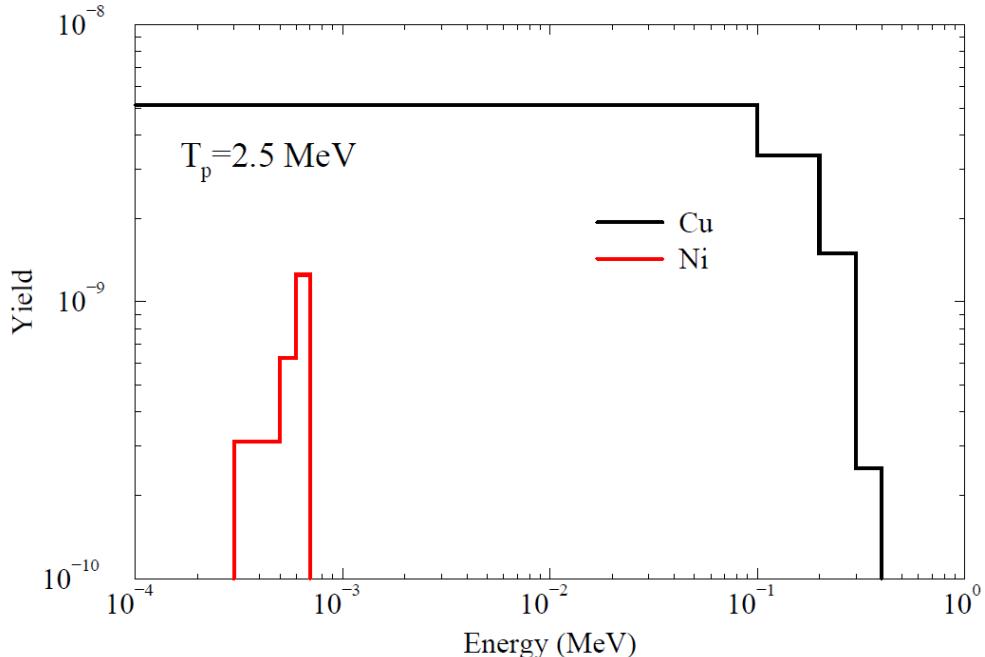
$$Y_n \approx \begin{cases} (6.6 - 26.4) \cdot 10^{-9} \text{ p/p} & \text{for Mo;} \\ 3.5 \cdot 10^{-9} \text{ p/p} & \text{for Cu.} \end{cases}$$

So, for molybdenum $\bar{Y} \approx 1.6 \cdot 10^{-8} \text{ p/p}$.

It is necessary to note, that this value is a very conservative: it was obtained under the assumption that the proton will fly the entire distance without changing its initial energy, so that the cross section of neutron production remains constant. In fact, the protons lose part of its energy in the initial part of its trajectory. This means that the neutron cross section will decrease (and significantly!), and as soon as the proton energy is below the threshold of the corresponding reaction, the number of neutrons will fall sharply.

Dose evaluation

To evaluate the dose it is necessary to know the energy spectrum of the radiated neutrons. It is shown for copper in the following figure [3] (these results were obtained with code MCNPX 2.6; by the way, the value of the neutron yield for copper agrees well with the above estimation):



It is seen that the maximal of neutron energy less than 400 keV, and the energy distribution of the neutrons is almost uniformly in the range from 0.1 keV till 100 keV, so that their average energy is about 50 keV.

Unfortunately, the similar spectrum data for molybdenum are not available. For these reasons let use for estimation data for copper: for these energies the dose per fluence conversion factor for neutrons $P_{eff}(E)$ is about $22 \text{ pSv} \cdot \text{cm}^2 = 22 \cdot 10^{-12} \cdot 10^5 = 2.2 \cdot 10^{-6} \text{ mrem} \cdot \text{cm}^2$ [5]. The dose is defined as

$$D_{[\text{mrem}/\text{hr}]} = \frac{P_{eff} [\text{mrem} \cdot \text{cm}^2] \cdot N_{neutron} [\text{hr}]}{S} = 3.6 \cdot 10^3 \frac{P_{eff} [\text{mrem} \cdot \text{cm}^2] \cdot \bar{Y} \cdot N_{proton}}{4\pi R^2},$$

where R is a distance from neutron source and number protons per second is determined with beam current I . For $I = 10 \text{ mA} \rightarrow N_{proton} \approx 6 \cdot 10^{16} \text{ sec}^{-1}$ and $R = 1 \text{ m}$ the very conservative estimation of dose is as follows:

$$D \approx \frac{3.6 \cdot 10^3}{4\pi \cdot 10^4} \underbrace{2.2 \cdot 10^{-6}}_{P_{eff}} \cdot \underbrace{1.6 \cdot 10^{-8}}_{\bar{Y}} \cdot \underbrace{6 \cdot 10^{16}}_{N_{proton}} \approx 60 \text{ mrem/hr.}$$

Again, this value should be regarded as a very conservative.

References

1. Q-value Calculator. <http://nndc.bnl.gov>
2. <http://www.nndc.bnl.gov/exfor/exfor00.htm>
3. N. Mokhov, I. Rakhno. "Neutron and Photon Production by Low Energy Protons". Fermilab-FN-0909-APC, December 2010.
4. STAR code. <http://physics.nist.gov/PhysRefData/Star/Text/PSTAR.html>
5. J.D. Cossairt, K. Vaziri. "Neutron Dose per Fluence and Weighting Factors for Use at High Energy Accelerators". Fermilab-Pub-08-244-ESH-REV, December 2008.